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## ОЦІНКА ДОЗОВОГО НАВАНТАЖЕННЯ ПРИ ДЕМОНТАЖІ РЕАКТОРА ВВР-М

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Реактор ВВР-М є гетерогенним дослідницьким реактором з легководним охолодженням і сповільнювачем з тепловою потужністю 10 МВт. Наразі триває остаточне планування зняття з експлуатації. Загальна стратегія зняття з експлуатації полягає в демонтажі та окремому вилученні громіздких елементів цілими без попередньої сегментації. Демонтаж первинного та вторинного контурів охолодження розглядається як одне з ключових завдань; розроблено проект окремого демонтажу. У даній роботі представлено основні принципи технічного рішення та безпеки. Результати дозової оцінки показали, що роботи можна виконати при колективній дозі менше 20 чол-мЗв.

*Ключові слова:* реактор типу ВВР, зняття з експлуатації, контури охолодження, демонтаж, доза опромінення.

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## ASSESSMENT OF THE DOSE LOAD DURING THE DISMANTLING OF THE WWR-M REACTOR

The WWR-M is a light-water-cooled and moderated heterogeneous research reactor with a thermal output of 10 MW. The final decommissioning planning is in progress now. The general decommissioning strategy consists of the dismantling and separate removal of the bulky elements as a whole (in one piece) without preliminary segmentation. The dismantling of the primary and secondary cooling loops is considered as one of the key tasks; a separate dismantling design has been developed. The baseline principles for the technical solution and safety are presented in the given paper. Results of the dose assessment showed that the work can be performed at a collective dose of less than 20 man-mSv.

*Keywords:* WWR type research reactor, decommissioning, cooling loops, dismantling, exposure dose.

## REFERENCES

1. General safety provisions for the decommissioning of nuclear installations. NP 306.2.230-2020. Registered with the Ministry of Justice of Ukraine on 30.12.2020 for No. 1311/35594. (Ukr)
2. Selection of Decommissioning Strategies: Issues and Factors. IAEA-TECDOC-1478 (Vienna, IAEA, 2005) 51 p.
3. Policies and strategies for the decommissioning of nuclear and radiological facilities. IAEA Nuclear Energy Series No. NW-G-2.1 (Vienna, IAEA, 2011) 43 p.
4. Planning, managing and organizing the decommissioning of nuclear facilities: lessons learned. IAEA-TECDOC-1394 (Vienna, IAEA, 2004) 111 p.
5. Preparing for Decommissioning During Operation and After Final Shutdown. OECD/NEA No. 7374 (Paris, 2018) 164 p.
6. Yu.N. Lobach, M.V. Lysenko, V.N. Makarovskiy. Substantiation of the decommissioning strategy selection for the research nuclear reactor WWR-M. *Nuclear and Radiation Safety* 12(4) (2009) 46. (Ukr)
7. Yu.N. Lobach et al. Progress in the decommissioning planning for the Kiev's research reactor WWR-M. *Nuclear Technology and Radiation Protection* 25(3) (2010) 239.
8. Yu.M. Lobach et al. Principal provisions of the decommissioning concept for the WWR-M reactor. *Yaderna Fizyka ta Energetyka (Nucl. Phys. At. Energy)* 22(4) (2021) 348.
9. S. Tozser. Full-scale reconstruction and upgrade of the Budapest research reactor. In: *Research Reactor Modernization and Refurbishment. IAEA-TECDOC-1625* (Vienna, IAEA, 2009) p. 83.
10. T.G. Apostolov, E.I. Anastasova, V.D. Anastasov. Implementation of the partial dismantling of research reactor IRT-Sofia prior to its refurbishment. *Nuclear Technology and Radiation Protection* 25(3) (2010) 249.
11. D. Kontogeorgakos et al. Occupational radiation exposure during removal of radioactive reactor components from GRR-1 pool. *Radiation Protection Dosimetry* 144 (2011) 632.
12. F. Meyer. Decommissioning of the ASTRA research reactor – planning, executing and summarizing the project. *Nuclear Technology and Radiation Protection* 25(3) (2010) 229.

13. K. Lauridsen, N. Strufe. Experience gained during the decommissioning of Danish research reactors DR 1 and DR 2. In: [Innovative and Adaptive Technologies in Decommissioning of Nuclear Facilities. Final report of a coordinated research project 2004 - 2008. IAEA-TECDOC-1602 \(Vienna, IAEA, 2008\) p. 129.](#)
14. A. Šimonis, P. Poskas, D. Grigaliuniene. Prediction of radiation doses during the dismantling of a maintenance cooling reservoir of RBMK-1500 reactor. [Progress in Nuclear Science and Technology 4 \(2014\) 824.](#)
15. Yu.N. Lobach, M.T. Cross. Dismantling design for a reference research reactor of the WWR type. [Nuclear Engineering and Design 266 \(2014\) 155.](#)
16. Yu.N. Lobach, G. Toth. Design for the WWR-M reactor vessel removal. [Nuclear Engineering and Design 258 \(2013\) 184.](#)
17. Yu.N. Lobach, V.N. Shevel. Pre-decommissioning complex engineering and radiation inspection of the WWR-M reactor. [Kerntechnik 79 \(2014\) 128.](#)
18. Yu.N. Lobach, E.D. Lufenko, V.N. Shevel. Radiation protection performance for the dismantling of the WWR-M primary cooling circuit. [Radiation Protection Dosimetry 162\(3\) \(2014\) 416.](#)
19. [Decommissioning Techniques for Research Reactors. Final report of a co-ordinated research project 1997 - 2001. IAEA-TECDOC-1273. \(Vienna, IAEA, 2002\) 268 p.](#)
20. [Decommissioning of Research Reactors: Evolution, State of the Art, Open Issues. IAEA Technical Reports Series 446 \(Vienna, IAEA, 2006\) 169 p.](#)
21. [Selection of Technical Solutions for the Management of Radioactive Waste. IAEA-TECDOC-1817 \(Vienna, IAEA, 2017\) 114 p.](#)
22. Yu.N. Lobach, V.N. Shevel. Design for the dismantling of the WWR-M primary cooling circuit. [International Nuclear Safety Journal 3\(4\) \(2014\) 25.](#)
23. [Managing Low Radioactivity Material from the Decommissioning of Nuclear Facilities. IAEA Technical Reports Series 462 \(Vienna, IAEA, 2008\) 213 p.](#)
24. [Decommissioning of Facilities. General Safety Requirements. IAEA Safety Standards Series. General Safety Requirements No. GSR Part 6 \(Vienna, IAEA, 2014\) 44 p.](#)
25. [Safety Assessment for the Decommissioning of Facilities Using Radioactive Material. IAEA Safety Standards No. WS-G-5.2 \(Vienna, IAEA, 2008\) 79 p.](#)
26. [Occupational Radiation Protection during the Decommissioning of Nuclear Installations. Main Aspects of Management, Planning and Conduct. IAEA-TECDOC-1954 \(Vienna, IAEA, 2021\) 65 p.](#)
27. Yu.N. Lobach, V.N. Shevel. Radiation protection tasks on the Kiev research reactor WWR-M. [Nuclear Technology and Radiation Protection 24\(2\) \(2009\) 145.](#)

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